

# Target Reliability as an Acceptance Criterion for Component Failure within a Systems Approach to Nuclear Structural Integrity Assessment

## 2<sup>nd</sup> International Seminar on Probabilistic Methodologies for Nuclear Applications

Mike Martin – Engineering Associate Fellow, Structural Integrity

Keith Wright – Engineering Associate Fellow, Structural Integrity and Chief Stress Engineer

25 October 2017, Ottawa

© 2017 Rolls-Royce plc

The information in this document is the property of Rolls-Royce plc and may not be copied or communicated to a third party, or used for any purpose other than that for which it is supplied without the express written consent of Rolls-Royce plc.

This information is given in good faith based upon the latest information available to Rolls-Royce plc, no warranty or representation is given concerning such information, which must not be taken as establishing any contractual or other commitment binding upon Rolls-Royce plc or any of its subsidiary or associated companies.

Trusted to deliver excellence



Rolls-Royce

Export Classification – Not Listed

# Content

- Background and Drivers
- Probabilistic Approaches
- Predictive Capability
- Systems Approach
- Summary



**Rolls-Royce**

Export Classification – Not Listed

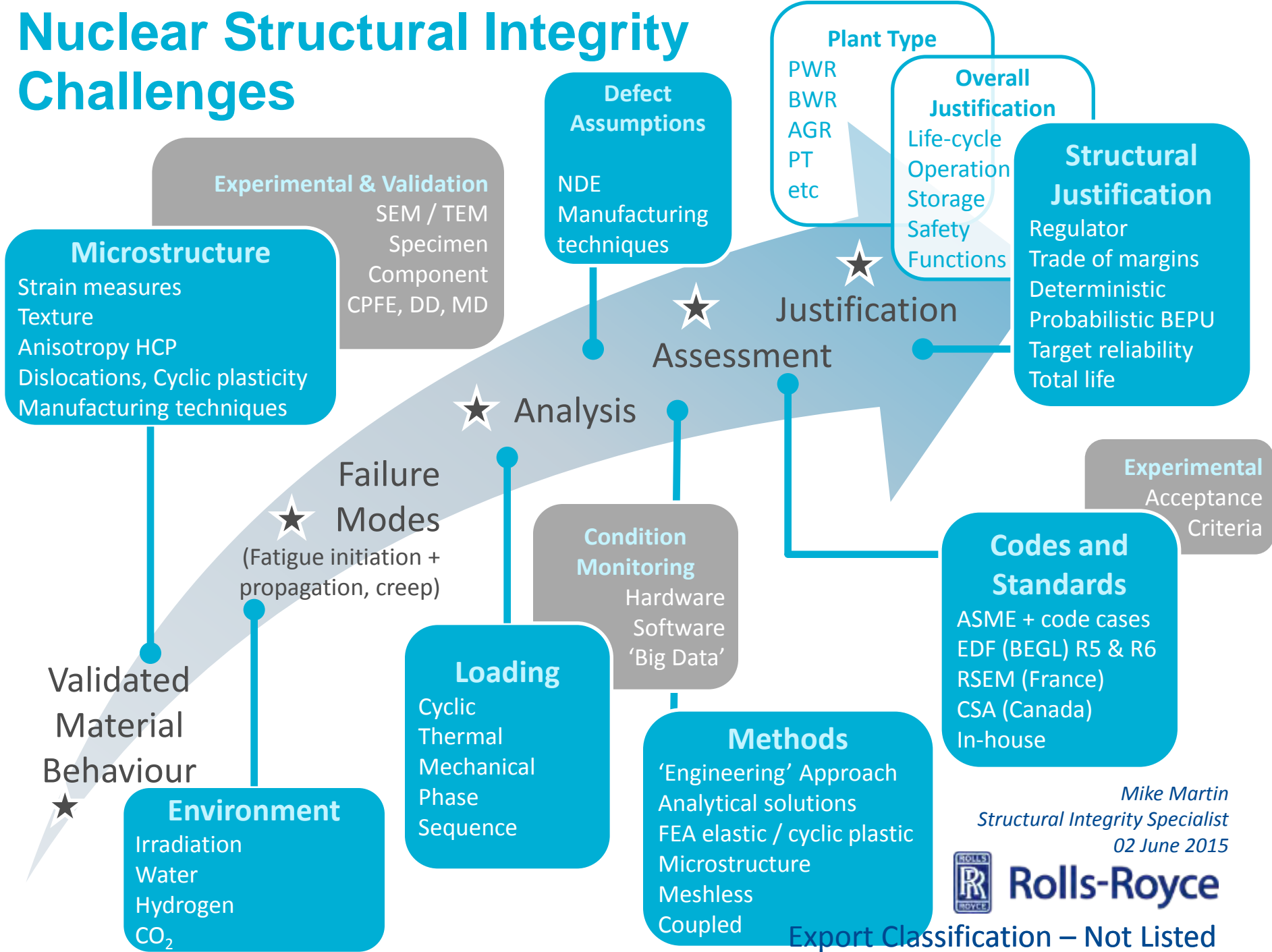
# Background and Drivers



**Rolls-Royce**

Export Classification – Not Listed

# Nuclear Structural Integrity Challenges

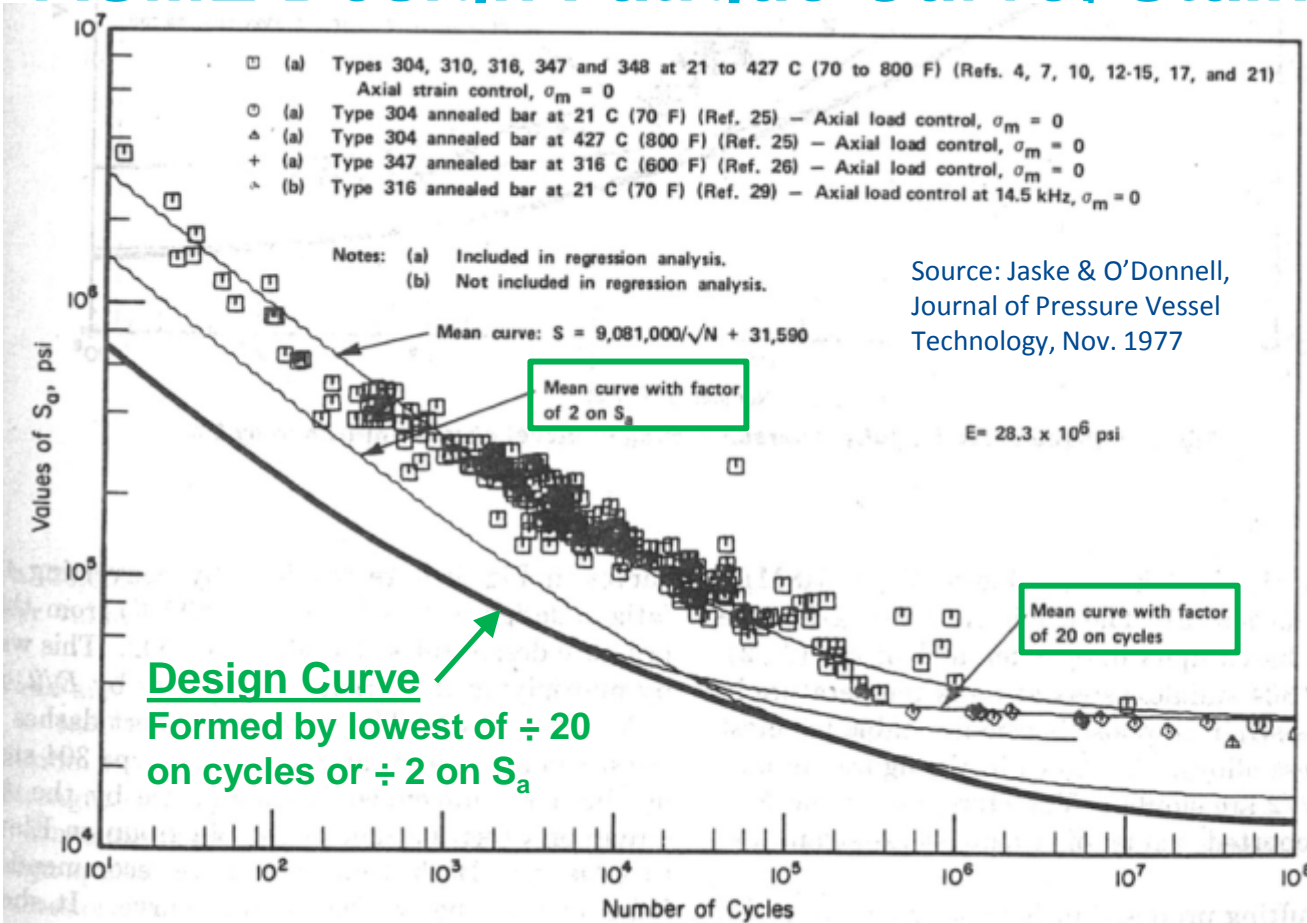


Mike Martin  
Structural Integrity Specialist  
02 June 2015



**Rolls-Royce**

# ASME Design Fatigue Curve: Stainless Steel



Source: Jaske & O'Donnell, Journal of Pressure Vessel Technology, Nov. 1977

Fig. 3 Fatigue curve for types 304, 310, 316, 347, and 348 austenitic steels

- Most test conditions:
  - Uniaxial strain-controlled loading
  - R = -1 (w.r.t. strain)
  - Plain specimens
  - Polished g.l. surface
  - Iso-thermal
  - Air environment
- “Factor of 20” made up of sub-factors for:
  - Data scatter
  - Size effects
  - Surface finish, “atmosphere”, etc.
- Design curve includes maximum mean stress correction (modified Goodman)

- Best fit to test data: “Langer equation”,  $\epsilon_a = A1.(N)^{-n1} + A2$
- Design curve was constant from ~1962 until 2010

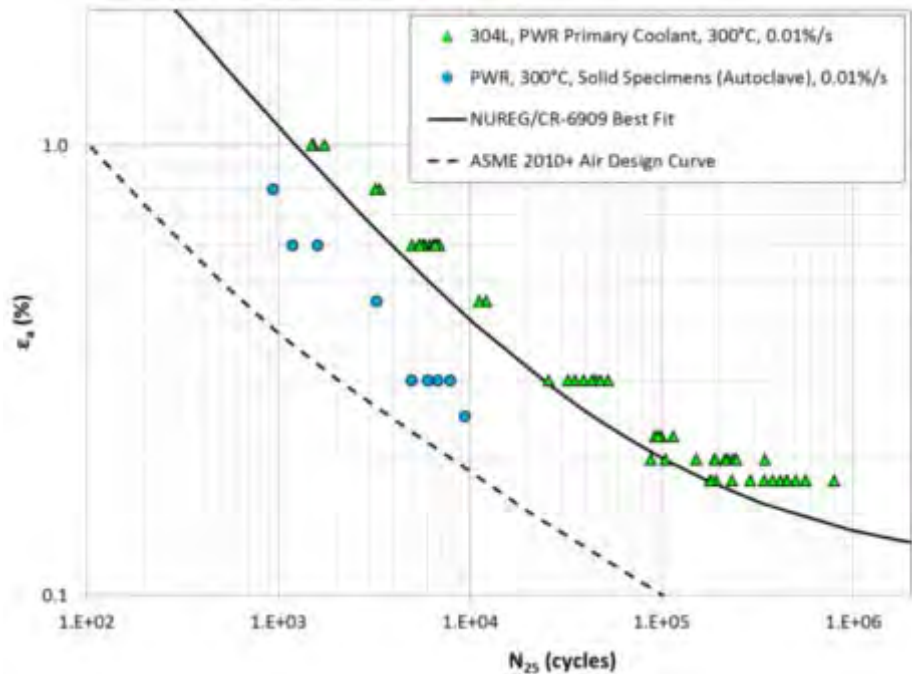
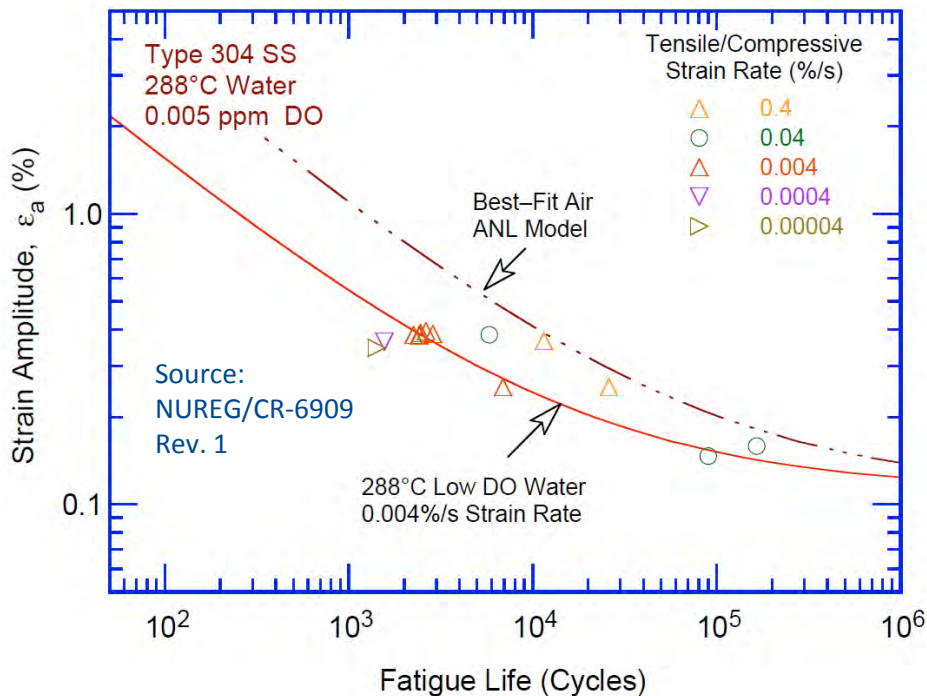
– Updated in 2010: updated best-fit to expanded database; factors of 12 & 2

– Changes based on US NRC report NUREG/CR-6909



Rolls-Royce

# Effect of PWR Environment



- **Environmental Correction Factor ( $F_{en}$ ) defined in NUREG/CR-6909:**

$$F_{en} = N_{air, RT} / N_{PWR} \quad \text{i.e. } N_{PWR} = N_{air, RT} / F_{en}$$

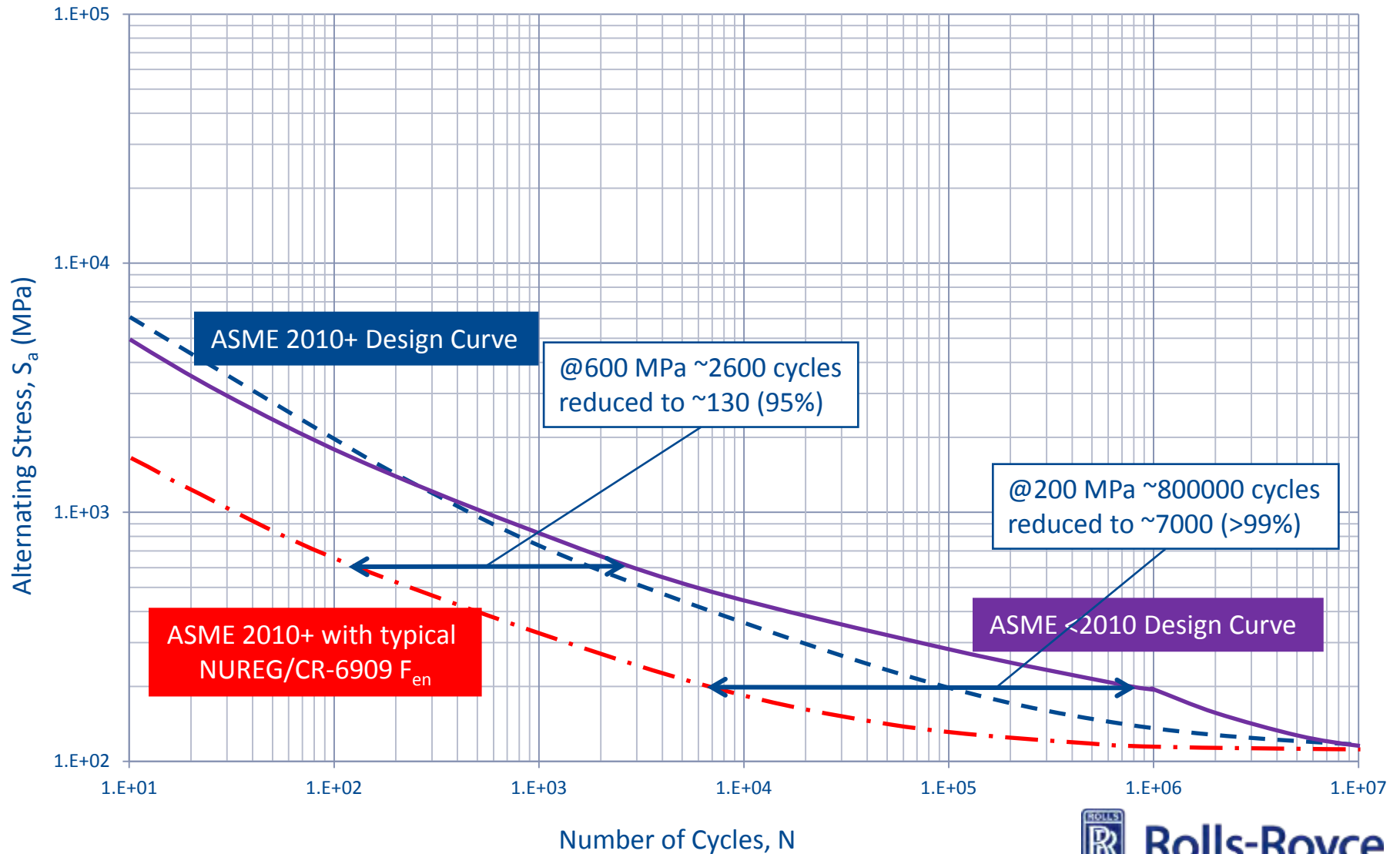
$$F_{en} = f(T, \dot{\epsilon}, \text{Dissolved Oxygen content})$$

- For austenitic stainless steels in PWR primary coolant (low DO),

$F_{en}$  function based on 200+ PWR test results

$$1.0 \leq F_{en} \leq 14.1$$

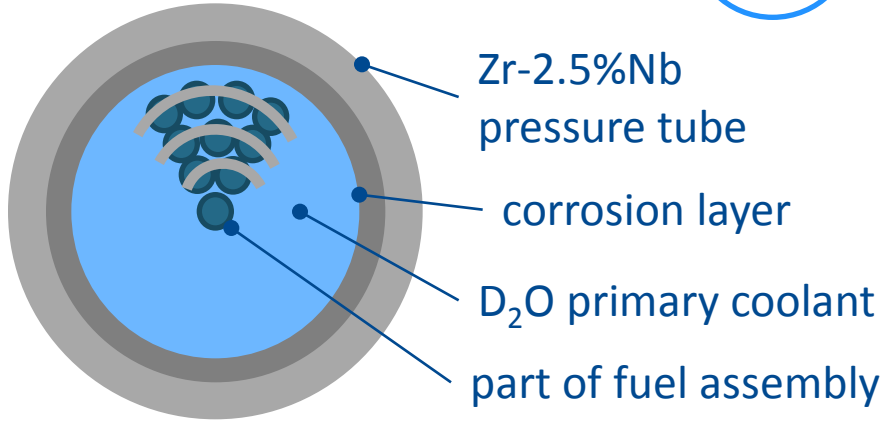
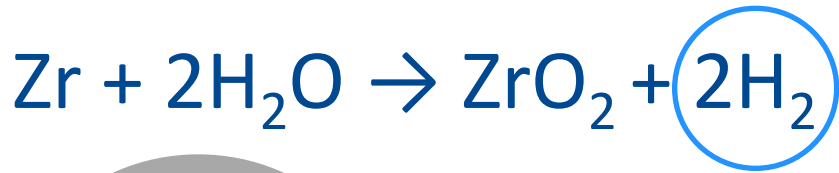
# Environmentally Assisted Fatigue - The Problem<sup>7</sup>



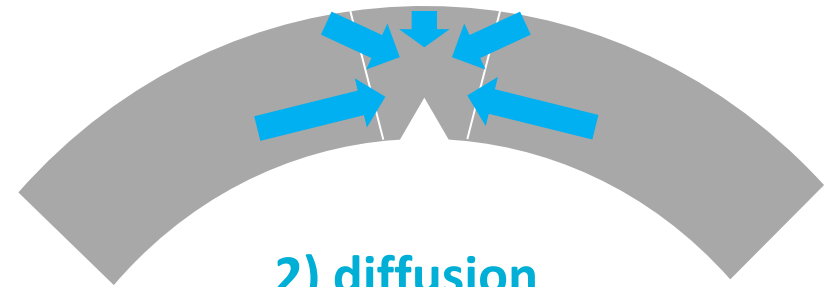
Rolls-Royce

Export Classification – Not Listed

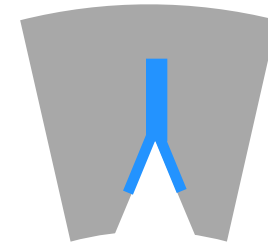
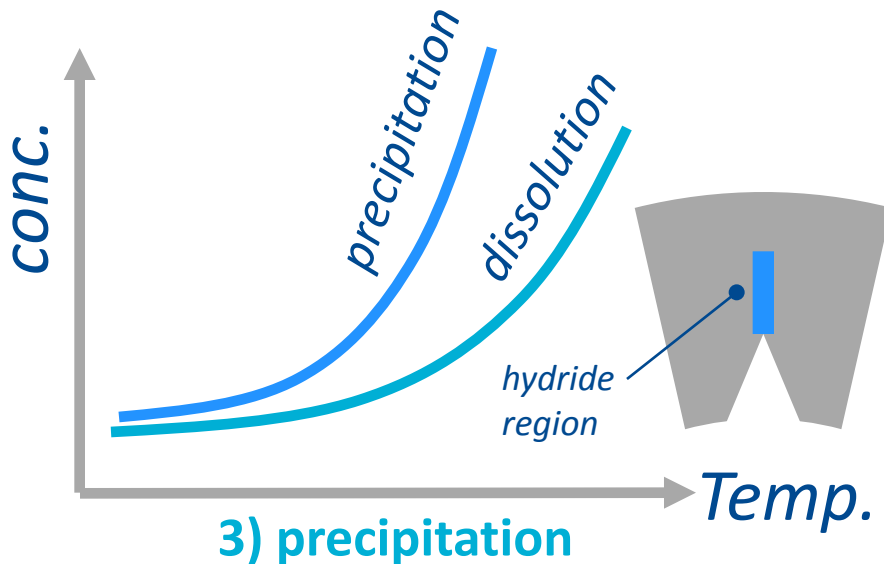
# DHC Mechanism



## 1) corrosion



Driven by gradients in stress, temperature and concentration



If load > threshold  $K_{IH}$ ,  
hydride cracks, stress concentration advances

## 4) cracking – self sustaining

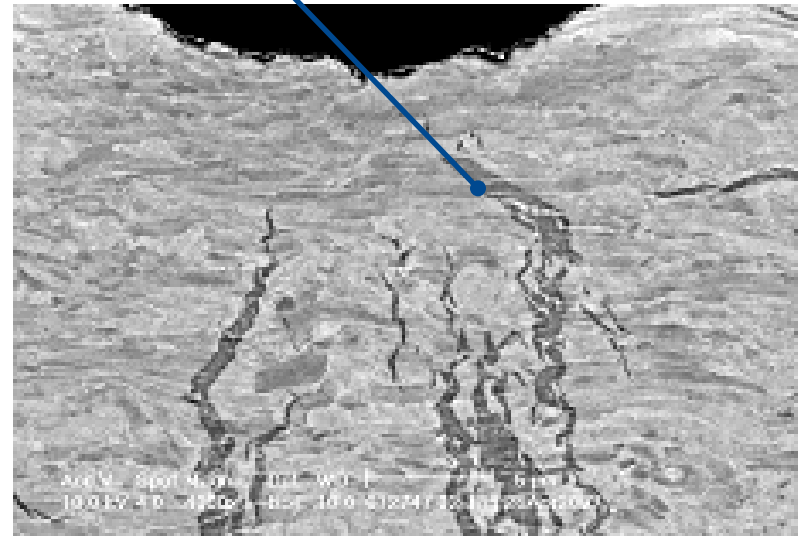
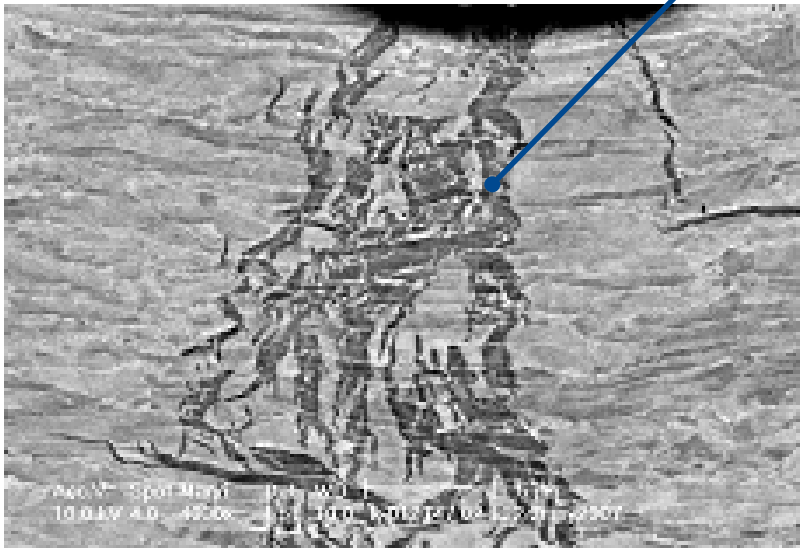


Rolls-Royce



# Some Hydride Distributions

hydride



Delayed Hydride Cracking in Zirconium Alloys – A Review of Mechanisms, Assessment Criteria and Current Developments

E Darby, M Martin and D Scarth

TAGSI / FESI Symposium 2013 Structural Integrity of Nuclear Power Plant



Rolls-Royce

Export Classification – Not Listed

# What is meant by 'Deterministic'?

A deterministic analysis will always produce the same output from a given set of initial conditions or inputs

No randomness

You get the answer in a pre-determined way for a particular set of inputs

## But 'Fully Deterministic' has become the norm!

- Engineers set **ALL** inputs to conservative if not bounding values
- The meaning of determinism is blurred, if not lost



Rolls-Royce

# International Atomic Energy Agency (IAEA) Commentary

Best Estimate Plus Uncertainty

*Whilst a best estimate approach to a deterministic analysis is permitted, provided that the uncertainties in the results are allowed for.....*

*It still requires that..... the remaining margins are adequate*

## Other Statements from UK Office for Nuclear Regulation (ONR)

...an adequate margin....

...suitably conservative....

...demonstrably conservative....

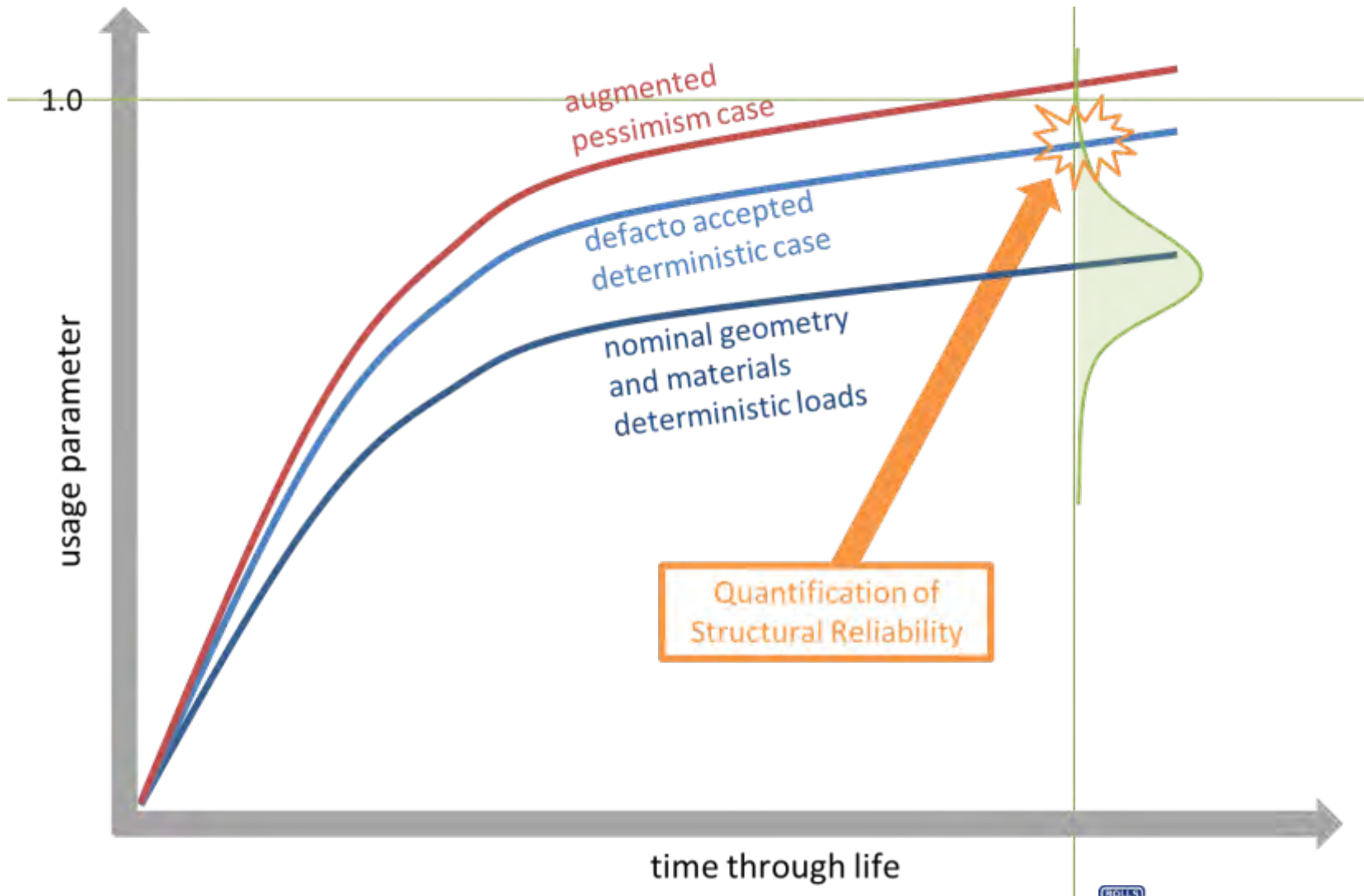
...very unlikely....

quantification required!



**Rolls-Royce**

# Setting the Scene....



**Rolls-Royce**

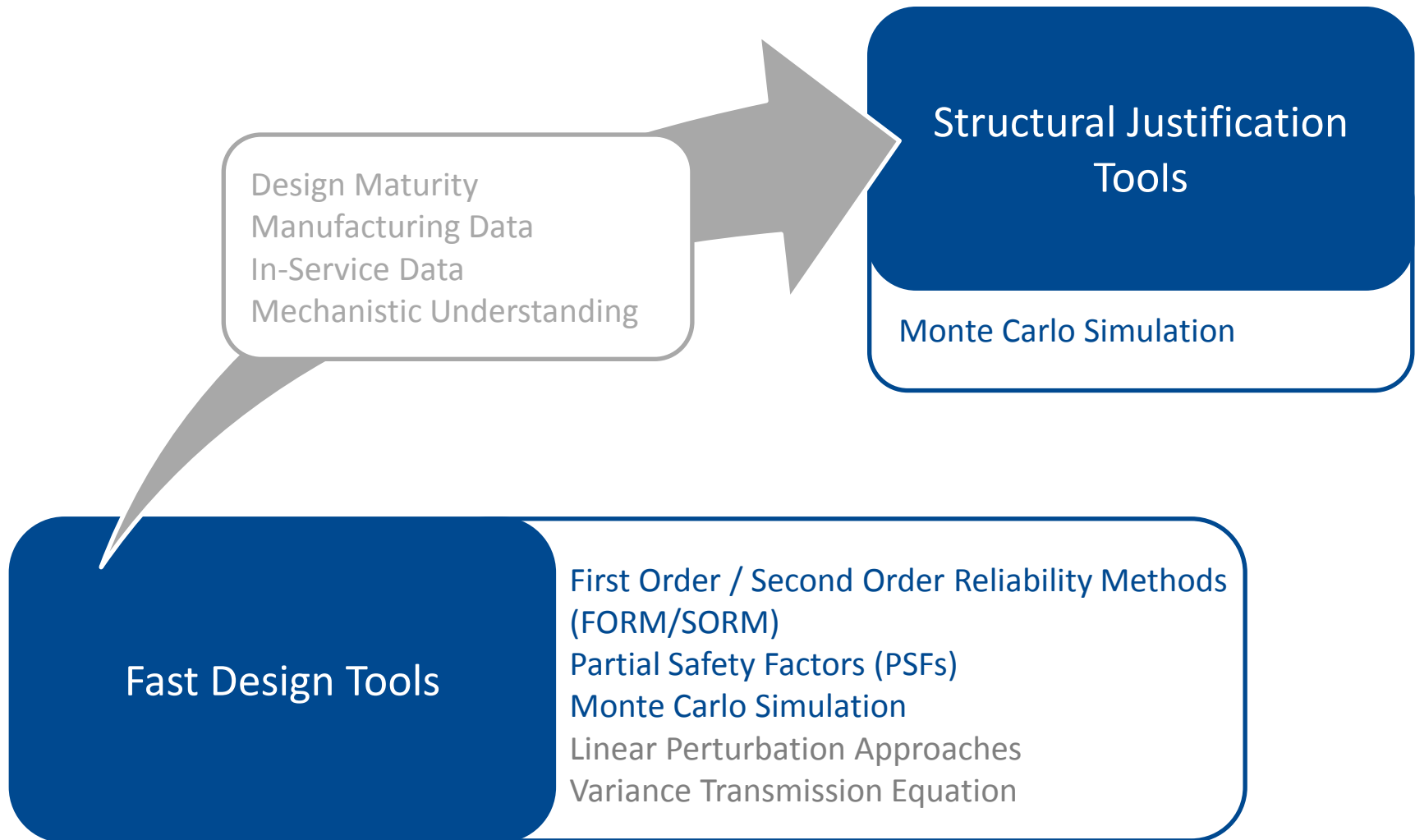
# Probabilistic Approaches



**Rolls-Royce**

Export Classification – Not Listed

# Hierarchy of Assessment Tools



# Partial Safety Factors (PSFs)

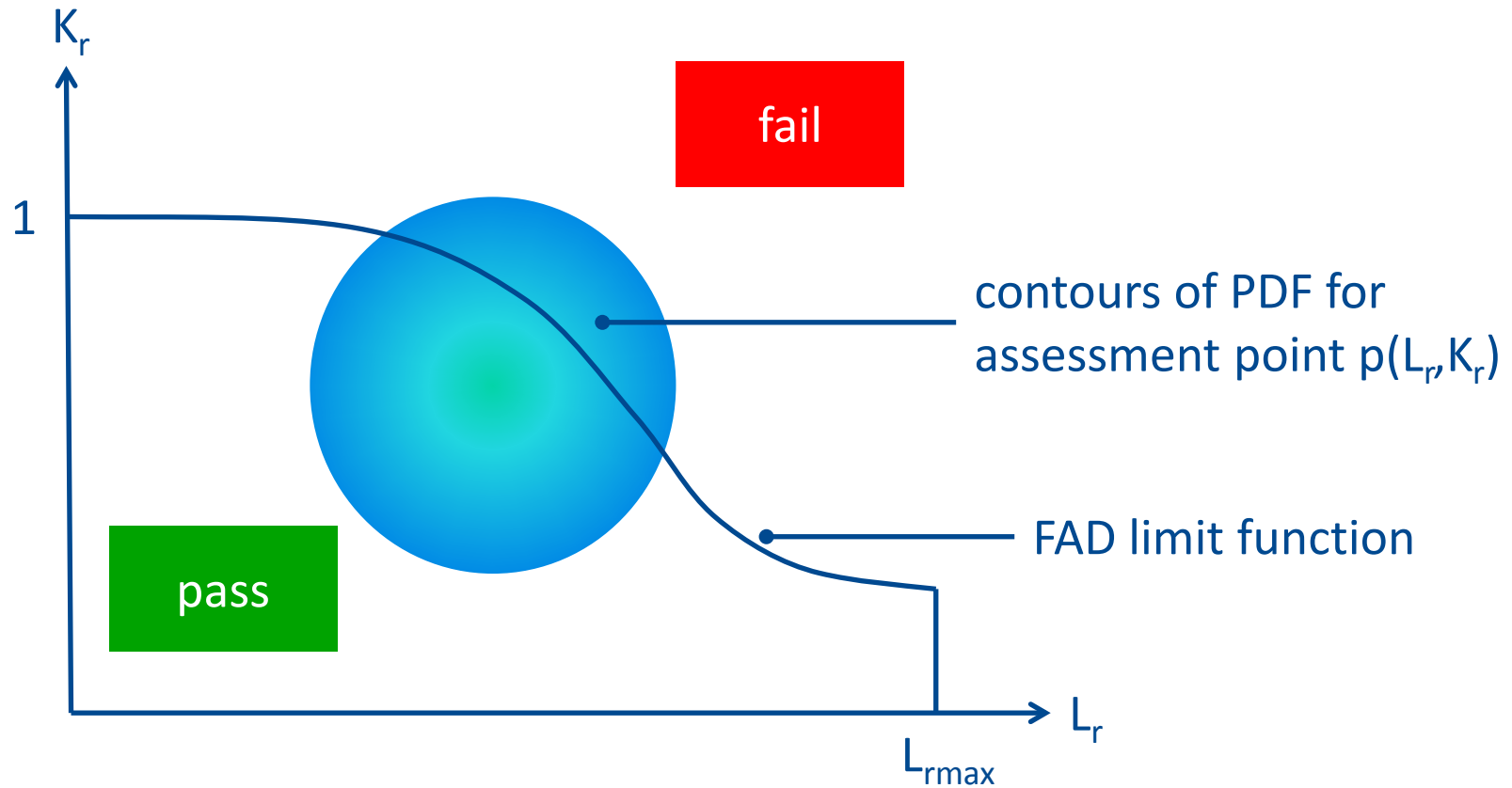
- Standard approach in structural design codes, eg BS7910, R6, ASME-FFS1/API-579, RSEM etc
- Applies tabulated factor to input data, resulting in margin to failure with quantified target reliability
- Calibrated using Monte Carlo simulation and FAD
- Rolls-Royce currently evaluating this approach with TWI (Cambridge UK)

Variable	COV	$2.3 \times 10^{-1}$ $\beta=0.739$	$10^{-3}$ $\beta=3.09$	$7 \times 10^{-5}$ $\beta=3.8$	$10^{-5}$ $\beta=4.27$	$10^{-7}$ $\beta=5.2$
Stress	....	....	....	....	....	....
Flaw size	0.1	1.0	1.4	1.5	1.7	2.1
	0.2	1.05	1.45	1.55	1.8	2.2
	0.3	1.08	1.5	1.65	1.9	2.3
	0.5	1.15	1.7	1.85	2.1	2.5
Toughness	....	....	....	....	....	....
Yield strength	....	....	....	....	....	....

Extract from  
BS7910 Table  
K.4

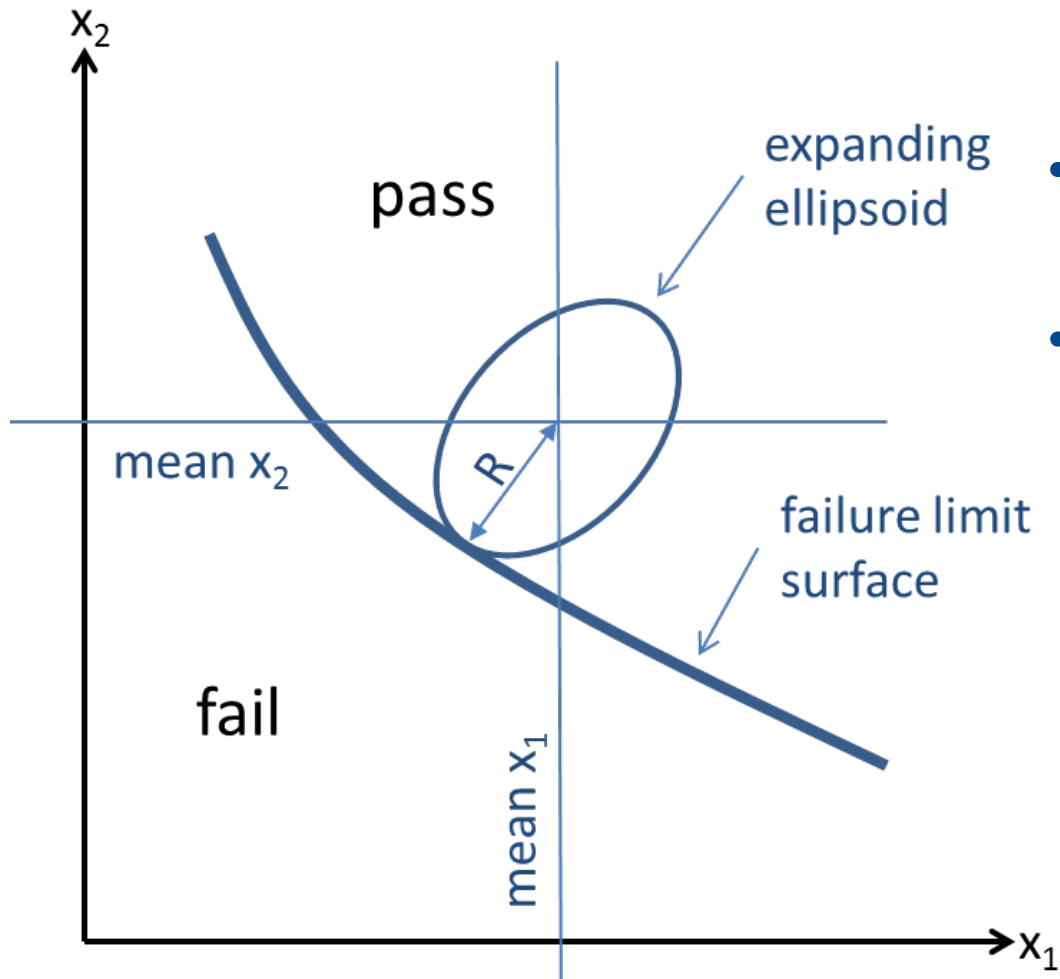


# Failure Assessment Diagram (FAD) Probabilistic Visualisation



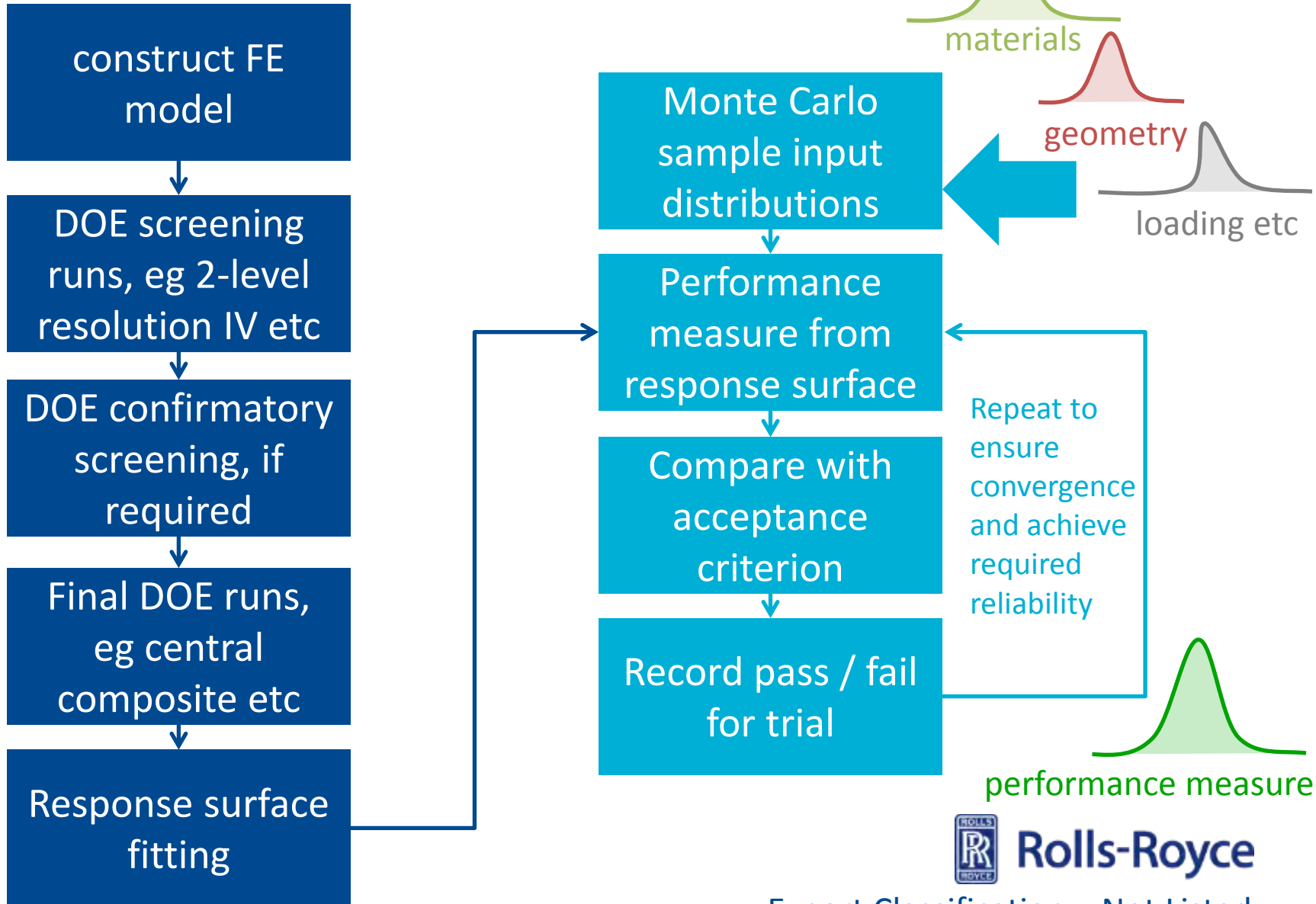


# First / Second Order Reliability Methods (FORM / SORM)



- Well established in oil and gas, aerospace, geotechnical....
- Assumption of normality
- Available in software and rapid spreadsheet applications possible in original variable space

# Typical Monte Carlo Approach



# Predictive Capability



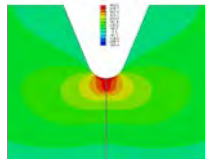
**Rolls-Royce**

Export Classification – Not Listed

# Development of DHC predictive capability

**Stage 1**  
process-zone  
CSA N285.8

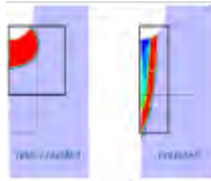
**Stage 2**  
process-zone /  
FE cohesive-zone  
arbitrary geometry



2016

**Stage 3**  
simple coupled  
cohesive-zone

transformation  
strain feedback

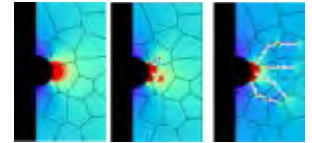


2017

**Stage 4**  
continuum  
coupled  
cohesive-zone

hydride failure  
behaviour

**Stage 5**  
coupled explicit  
Microstructural  
microstructural  
influence



2021

basic model data, eg KIH

hydride cohesive behaviour

TSS and precipitation

microstructure

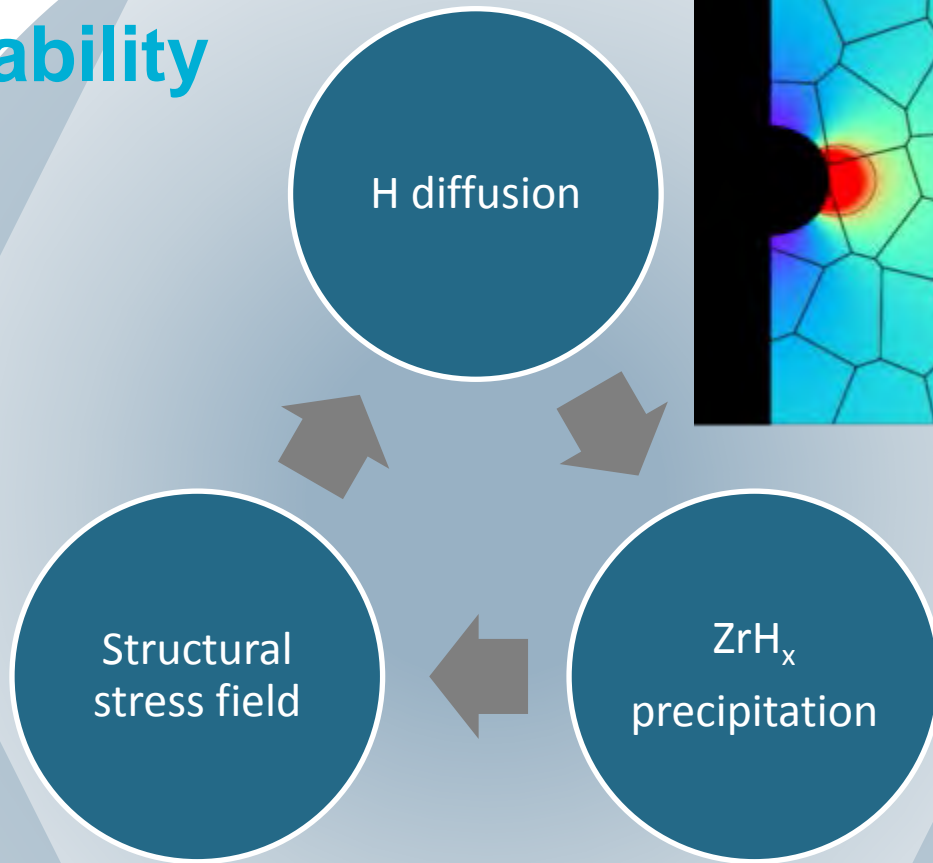
**Data  
requirements**



**Rolls-Royce**

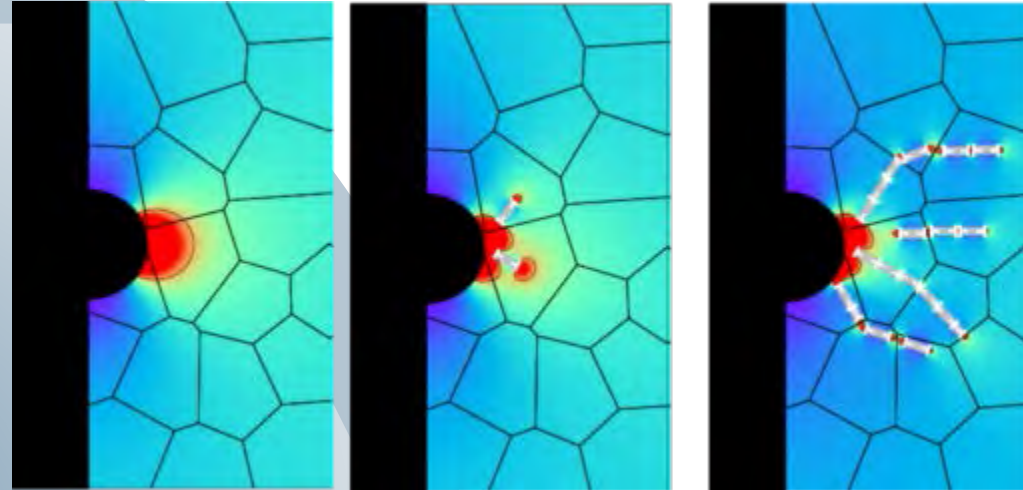
Export Classification – Not Listed

# Coupled predictive capability



microstructure

mechanical and thermal load cycle



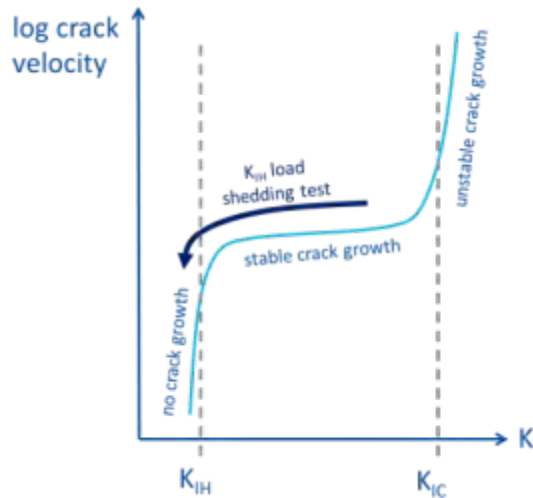
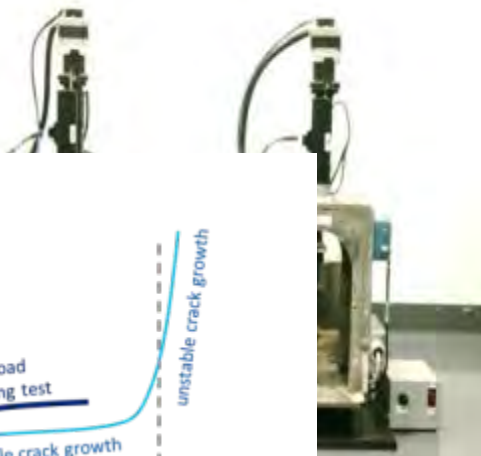
M Patel, Imperial College, 2015  
Multiscale modelling of DHC



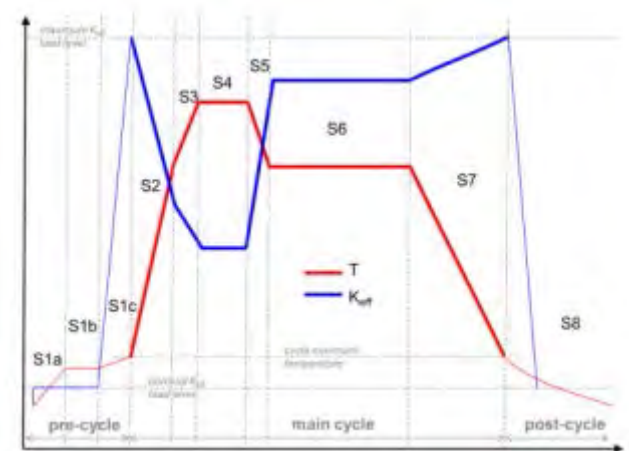
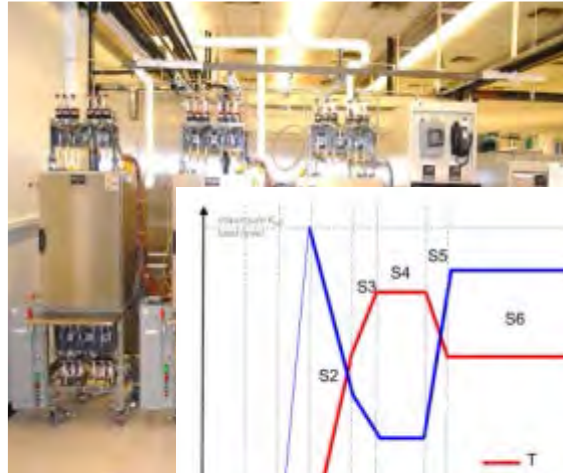
Rolls-Royce

# Validation Testing at Canadian Nuclear Laboratories (CNL)

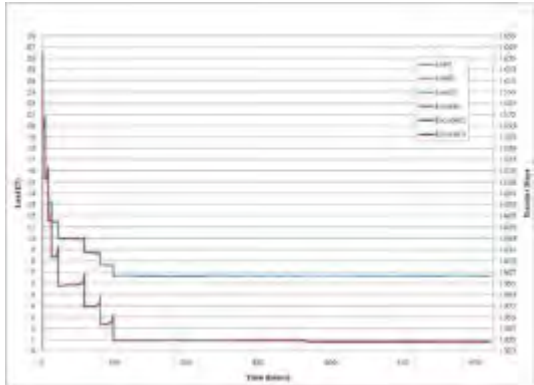
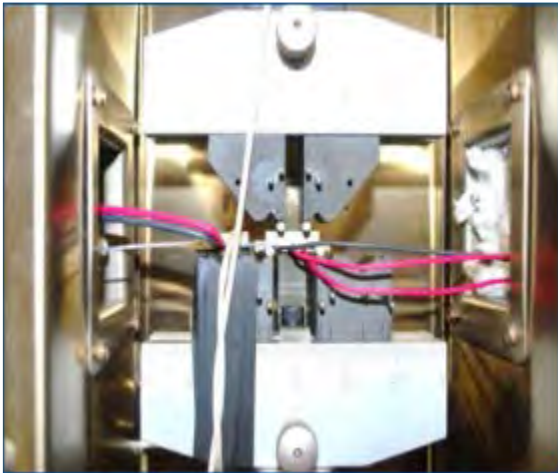
## KIH Rigs



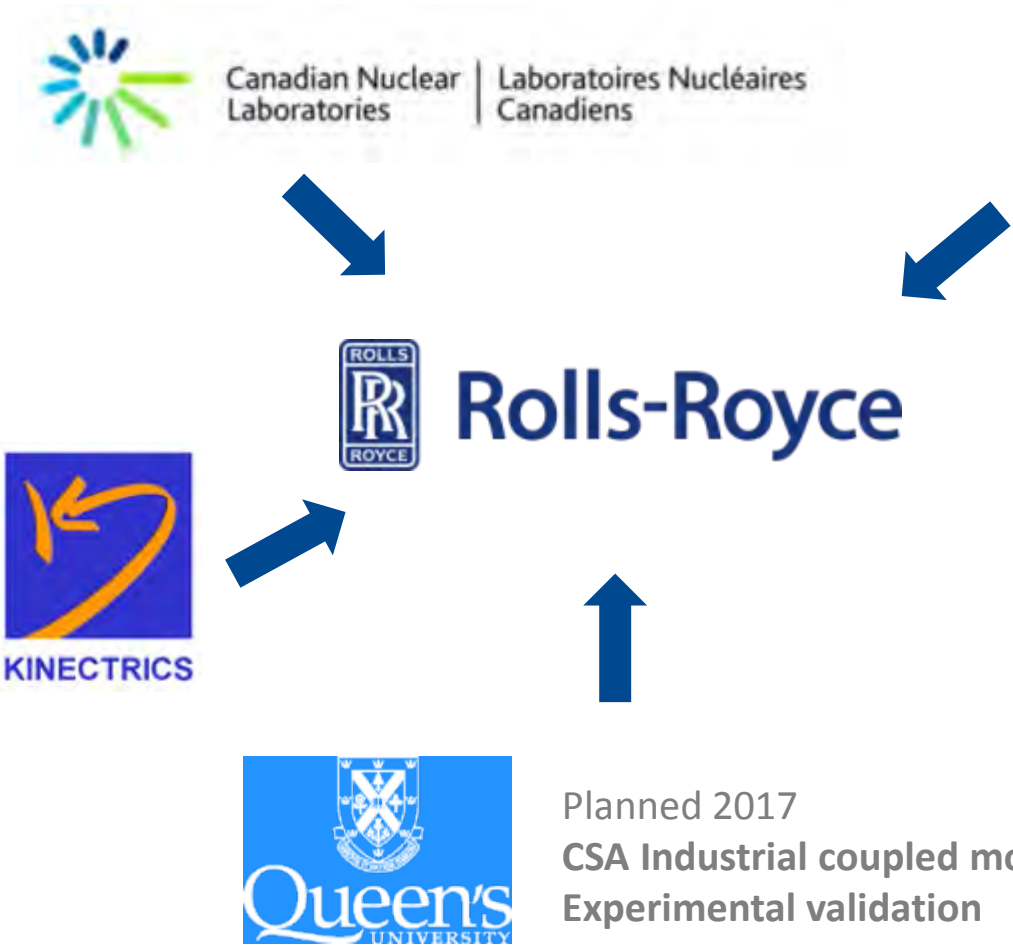
## Cycling DHC Rigs



## Fatigue Initiation Rigs



# Wider Industrial and Academic Network



 **Rolls-Royce**  
Nuclear UTC

**Imperial College**  
London



Mitesh Patel 2014  
**Multiscale modelling of DHC**  
David Wilson 2015  
**Mechanistic fatigue initiation model**  
Said el Chamaa 2016  
**DHC Characterisation**



The University of Manchester

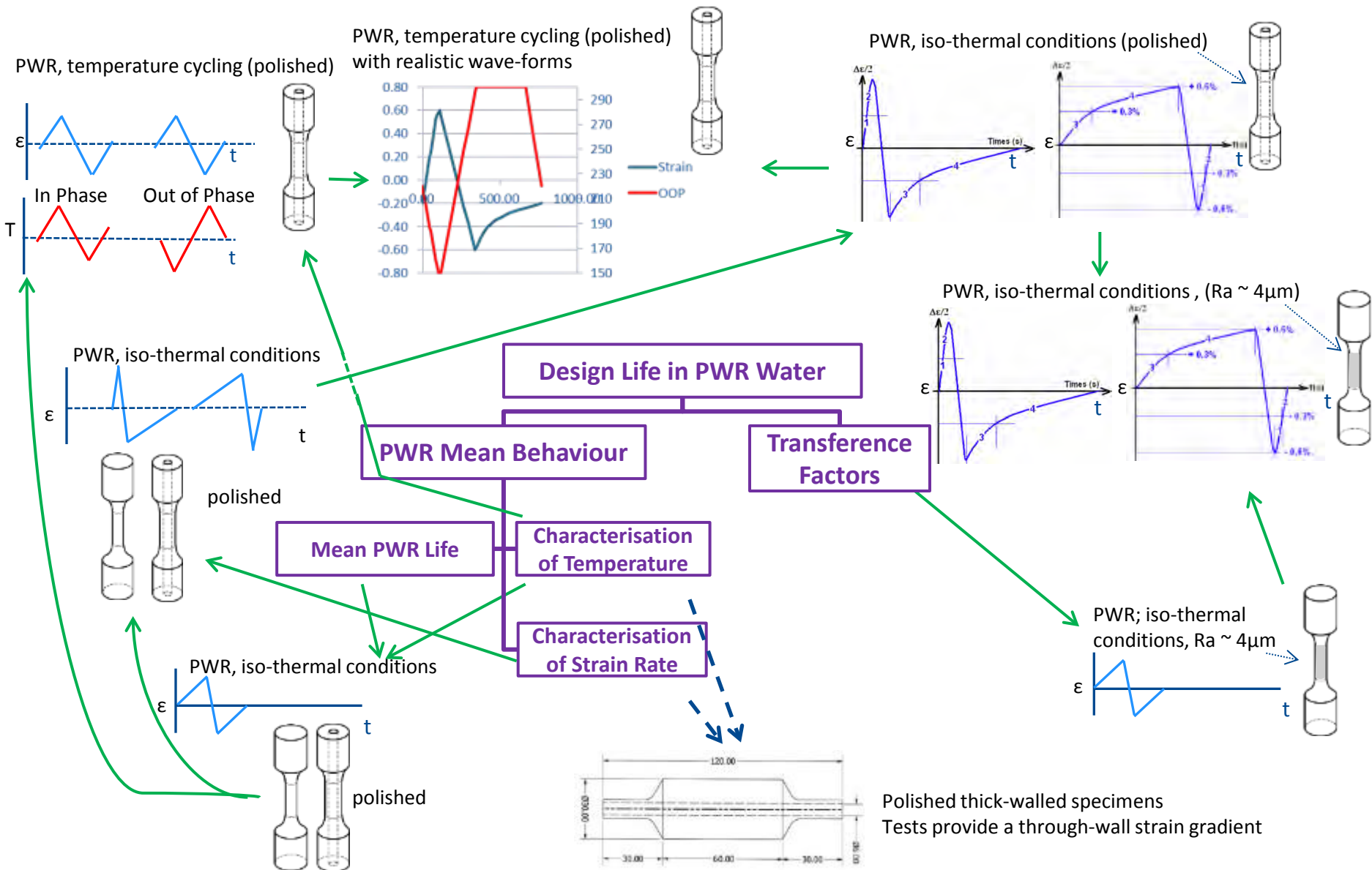
Planned 2017  
**Further characterisation**  
work to support models

 **Rolls-Royce**

Export Classification – Not Listed



# Testing of Plant-Realistic Loading



Polished thick-walled specimens  
Tests provide a through-wall strain gradient



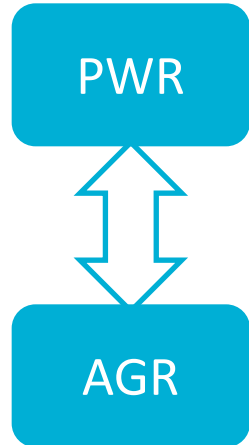
# A Way Forward – Total Life Prediction

- Synergy with EDF R5 high temperature approach, particularly at strain gradient
- Life to leakage apportioned:
  - Nucleation / Initiation
  - Short growth
  - LEFM fatigue crack growth
- Consequence based reliability target and probabilistic consideration of the inputs required .

$$N_{initiation} = A(\Delta\varepsilon)^b$$

$$\frac{da}{dN} = B(\Delta\varepsilon)^c$$

$$\frac{da}{dN} = C(\Delta K)^d$$



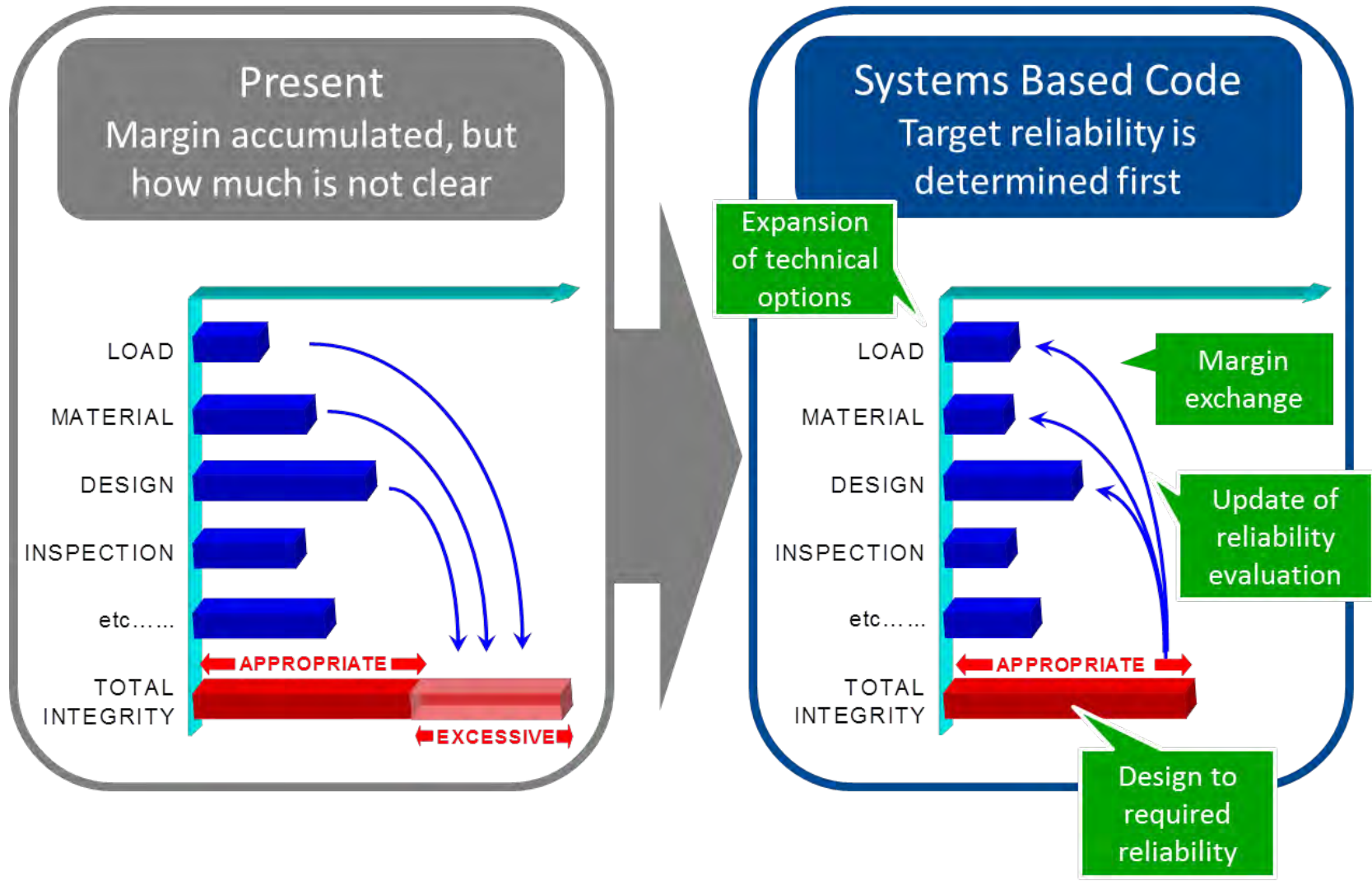
# Systems Approach



**Rolls-Royce**

Export Classification – Not Listed

# Systems-Based View



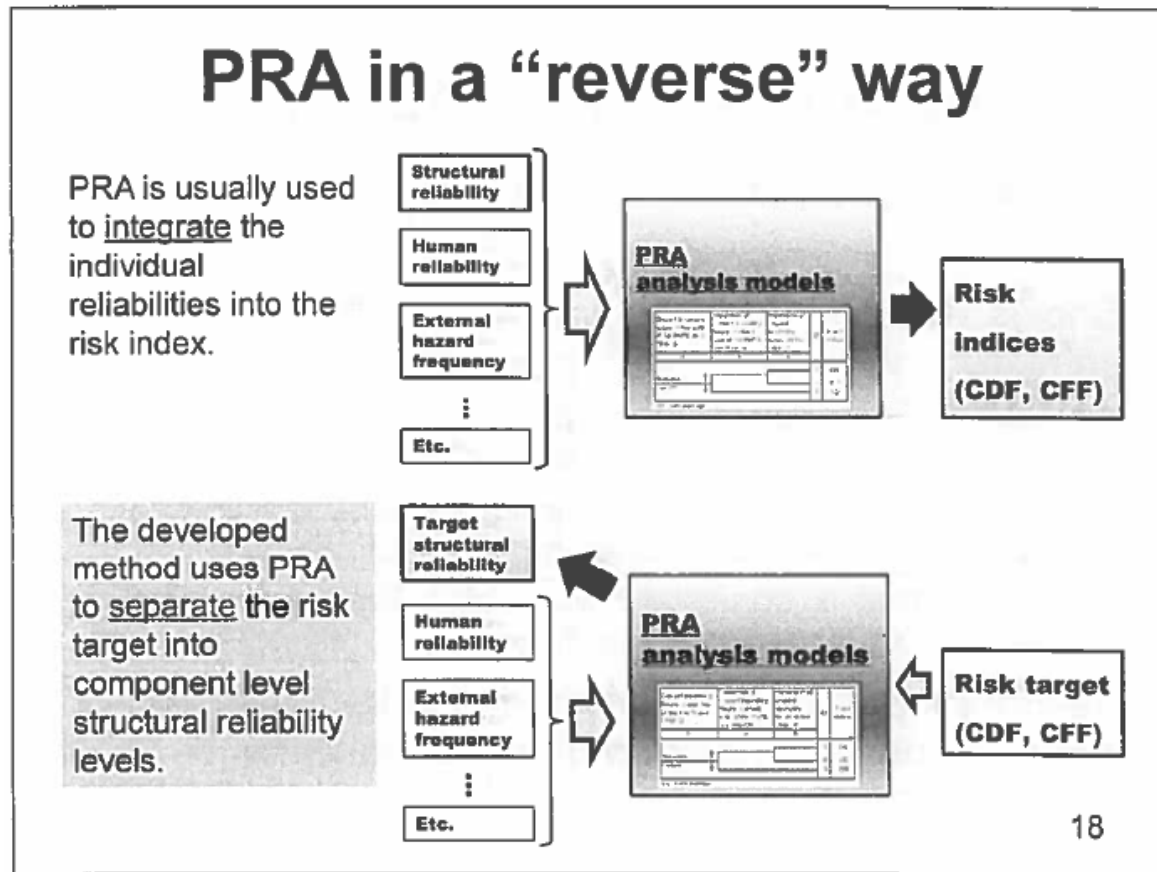
Adapted from Systems Based Code – Principal Concept  
Y Asada et al, Proceedings of 10<sup>th</sup> International Conference on Nuclear  
Engineering, 2002



**Rolls-Royce**

# Proposed Derivation of Target Reliabilities

- Therefore need to be better informed and work back from the core damage frequency targets / plant level safety criteria
- Use of PSA fault tree & Proportion of risk allocated to each system



JSME proposal and ASME Section XI RIM Task Group

Still a need to allow for uncertainties

# Target Reliabilities as Acceptance Criteria

- Assuming our predictive capability is sufficiently developed and validated by testing, what then?
- An acceptable **Fully Deterministic** approach is unlikely for limiting components due to number of inputs involved
- A probabilistic consideration of inputs is required to establish a margin to the best estimate predicted total life. That margin needs to exceed the target reliability derived from an understanding of the contribution to core damage
- Calibrate partial safety factors to achieve the reliability required



# Summary



**Rolls-Royce**

Export Classification – Not Listed

# Summary 1

- Higher fidelity modelling provides improved mechanistic understanding of key failure modes of industrial importance
- Coupled analysis paves the way for more integrated analysis approach and improved understanding of contribution to overall reliability
- Academic and wider industry network essential to delivery, as is IT infrastructure
- Regulator engagement throughout is key



**Rolls-Royce**

## Summary 2

- The traditional ASME one-size fits all approach to fatigue is **too conservative**. Deterministic approaches hide behind unquantified margins – not tolerable when compounded as with environmental effects
- We are more informed about environmental effects, certainly on laboratory specimens, but to claim benefit we need **Total Life Prediction**
- Mechanistically informed predictive capability for nucleation and short crack behaviour is required to demonstrate **target reliabilities**
- We need to translate plant level safety criteria to system and component target reliabilities: **Understanding target reliabilities is still some way off !**



Rolls-Royce



**Thank You for Listening!**



**Rolls-Royce**

Export Classification – Not Listed